

Shielding and Activation Issues for ARIES-AT

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Web address:
<http://fti.neep.wisc.edu/FTI/ARIES/AUG99/shield.pdf>

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Contents

- Updated Shielding requirements
- Optimal shield design
- Recommended radial/vertical builds for SiC/LiPb system
- Impact of magnet cryogenic shield on radial standoff
- Comparison between afterheat of WC and FS-based IB shields
- Clearance issues and recommended changes to V.V.

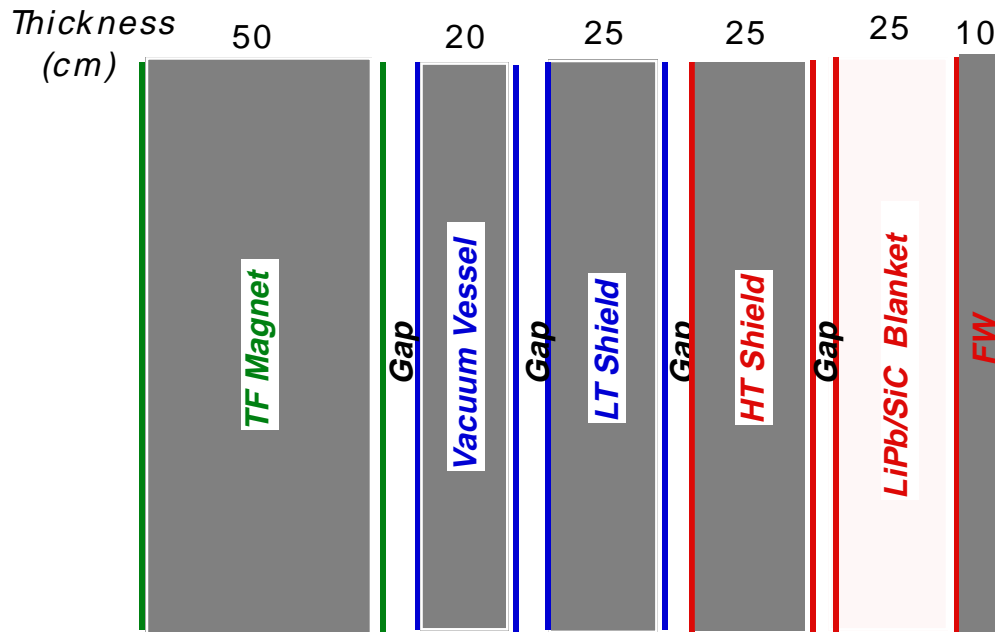
Shielding Requirements

- Provide lifetime protection for HT magnets $< 10^{19}$ n/cm²
- Provide lifetime protection for V.V. < 1 He appm
- Protect workers/personnel during operation < 2.5 mrem/h
- Power production component
($< 1\%$ nuclear heating in LT shield)
- OB shield is lifetime component < 200 dpa for FS
 $< 3\%$ burnup for SiC
- Reasonable cost
- Attractive safety & environmental characteristics:
 - Low level waste (Class C)
 - No hazardous materials
 - No damage in case of LOCA/LOFA
- Clear as many components as design allows for reasonable cost
- Meet stress and temperature limits
- Reliable, maintainable, replaceable, recyclable

Main Features of Shield/V.V.

- High performance, expensive materials for **IB side**
- Lower performance, inexpensive materials for **OB side**
- **LiPb-cooled HT shield with SiC** structure for **self-cooled** design
- **He-cooled (?) HT shield with SiC** structure for **dual-cooled** design
- **H₂O-cooled LT shield and V.V. with FS** structure
- ARIES-RS' **V.V.** configuration (20 cm IB, 20 cm div., 30 cm OB)

Inboard Radial Build



Components

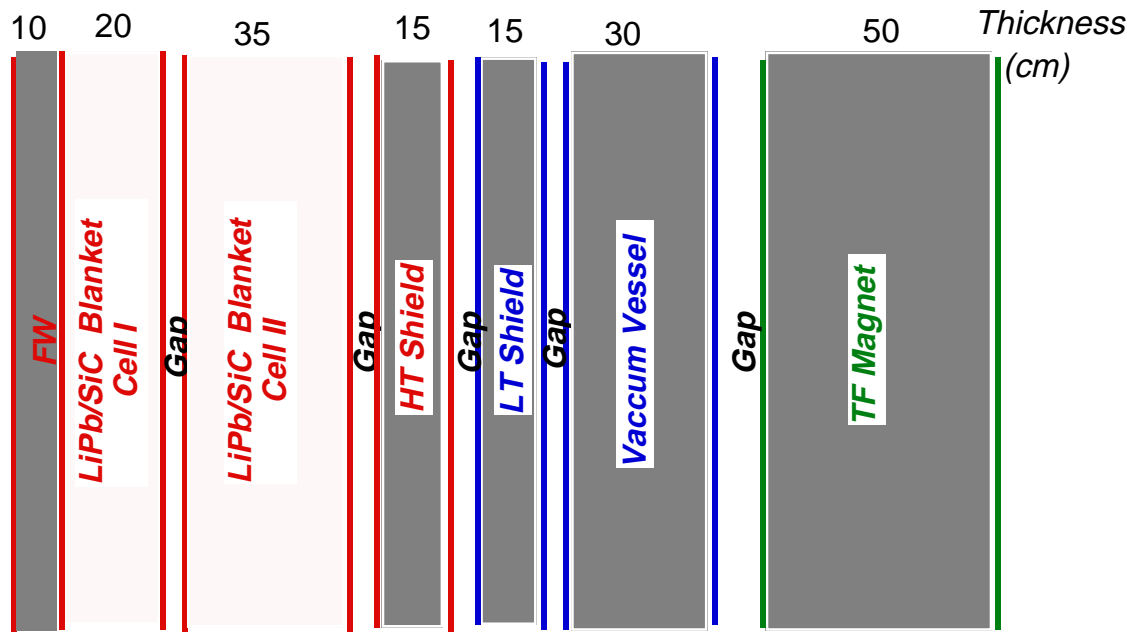
Composition

FW	17% SiC , 26% LiPb, 57% void
Blanket	8% SiC , 92% LiPb
HT Shield	15% SiC, 10% LiPb , 75% B-FS
LT Shield	15% FS , 5% H ₂ O , 80% WC
Vacuum Vessel	35% FS , 40% H ₂ O , 25% WC

- **Shield** sized for self-cooled FW/B design
- **No significant difference** in total FW/B/S/VV thickness between self-cooled and dual-cooled designs
- V.V. and TF magnet **radiation limits** are all met* for peak $\Gamma = 5 \text{ MW/m}^2$
- Higher wall loading requires thicker LT shield
- **Old LT magnet info** used for shielding analysis (**need info on HT magnet**)

* Safety factor of 3 considered in all shielding calculations

Outboard Radial Build



Components

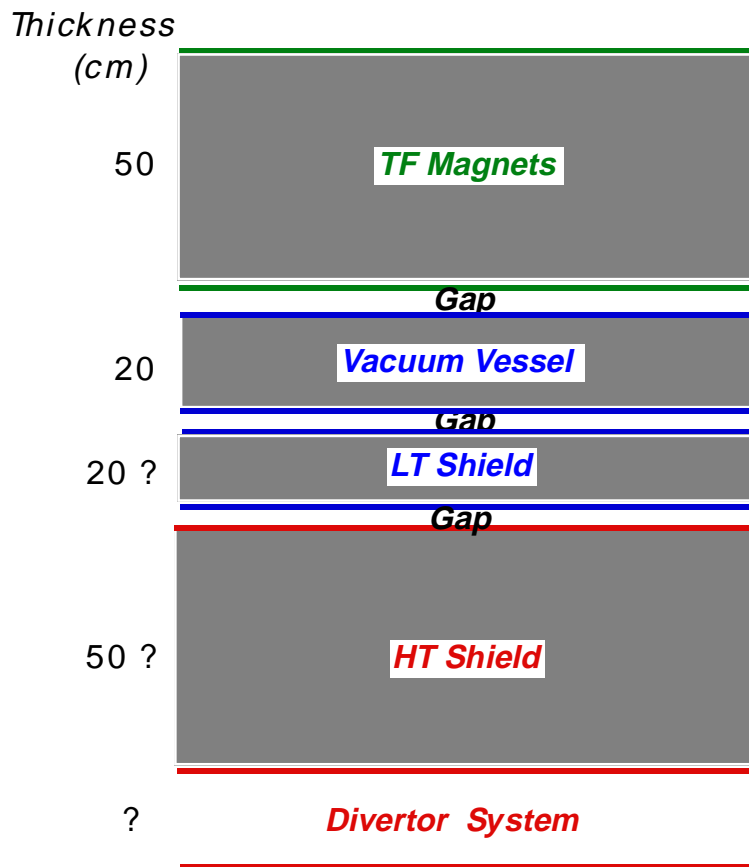
Composition

FW	17% SiC , 26% LiPb, 57% void
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HT Shield	15% SiC, 10% LiPb , 75% B-FS
LT Shield	15% FS , 5% H ₂ O , 80% B-FS
Vacuum Vessel	25% FS , 60% H ₂ O, 15% B-FS

- **Shield** sized for self-cooled FW/B design
- **Blanket Cell II and HT shield could be combined** in a single lifetime component
- **No significant difference** in total FW/B/S/VV thickness between self-cooled and dual-cooled designs
- V.V. and TF magnet **radiation limits** are all met* for peak $\Gamma = 7 \text{ MW/m}^2$
- Higher wall loading requires thicker LT shield
- Old LT magnet info used for shielding analysis (need info on HT magnet)

* Safety factor of 3 considered in all shielding calculations

Vertical Build



Divertor System

?% SiC , ?% LiPb or He

HT Shield

15% SiC , 10% LiPb , 75% B-FS

LT Shield

15% FS , 5% H₂O , 80% B-FS

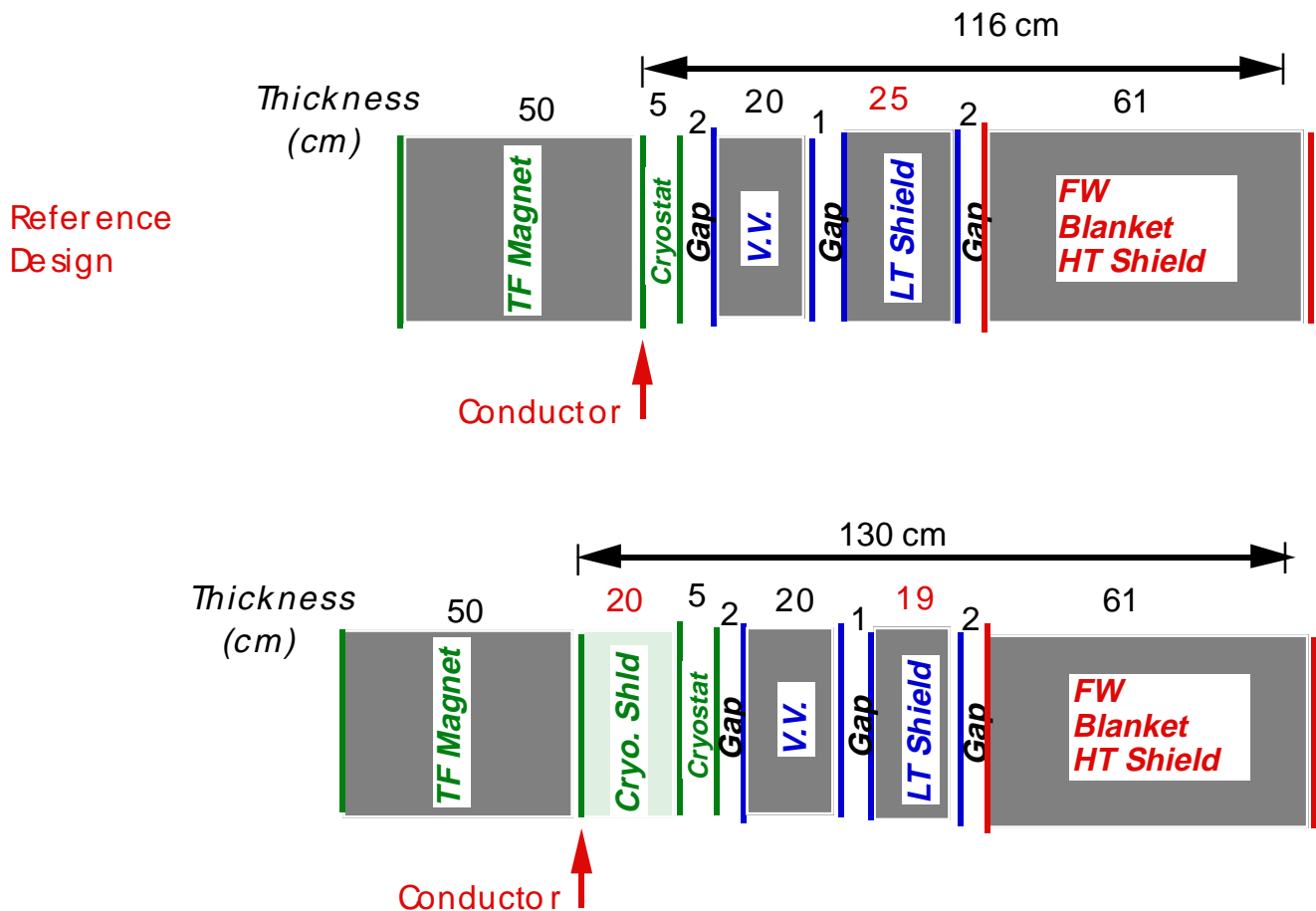
Vacuum Vessel

35% FS , 40% H₂O , 25% B-FS

- Shield size depends on divertor system design

Impact of Magnet Cryogenic Shield on Radial Standoff

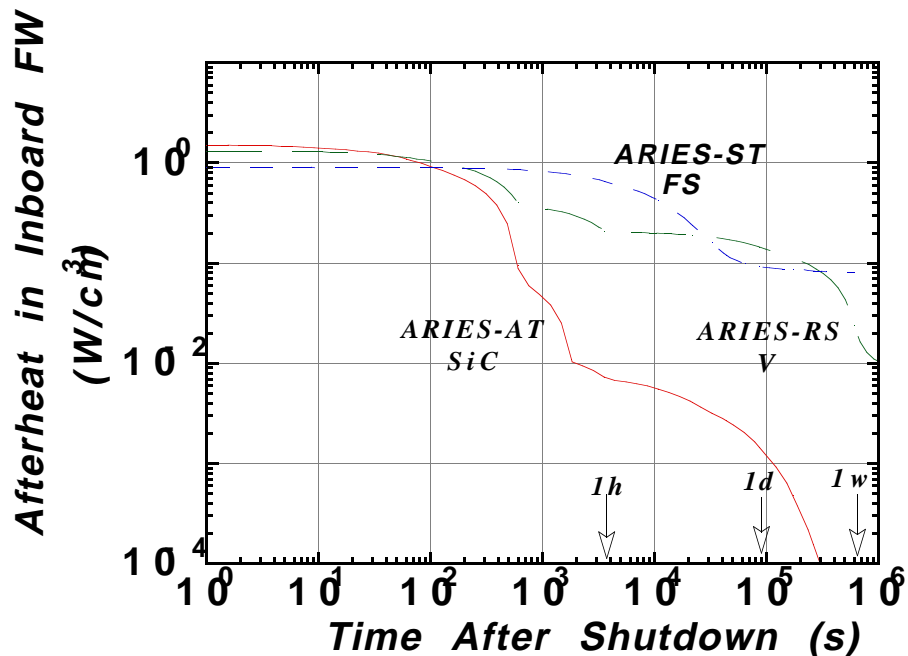
- Water-cooled WC-based IB shield/V.V. is much more efficient than SS-based magnet cryogenic shield
- Using 20 cm cryogenic shield reduces IB LT shield thickness by 6 cm but increases FW-conductor distance by 14 cm
- Cryogenic shield will not reduce radial standoff between FW and conductor



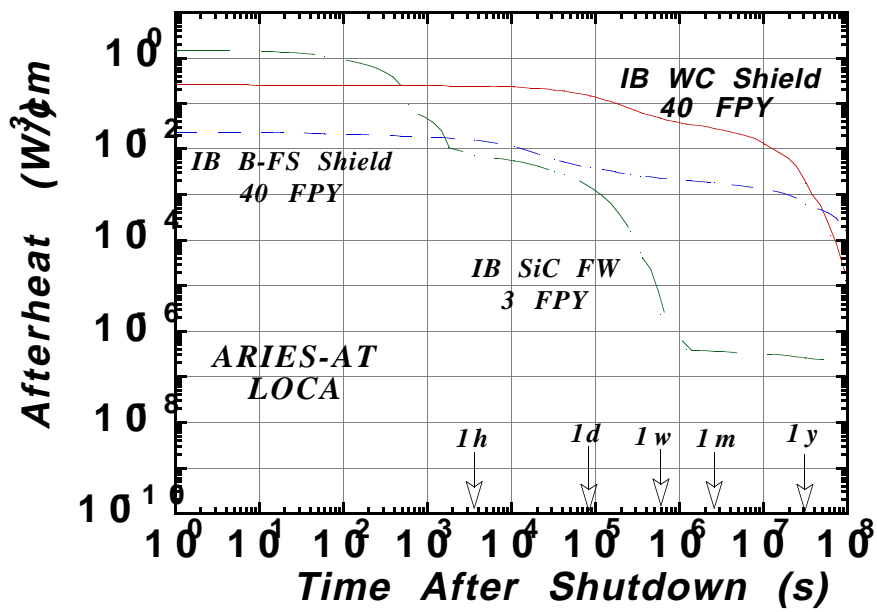
Cryogenic SS shield is not recommended for HT magnets

Activation Analysis

- **Codes and model:**
 - Activation: **ALARA** code; **FENDL-2** activation library
 - Flux: 1-D transport **DANTSYS** code; FENDL-1 Xn data; 175 n and 42 g group structure
 - **3-D neutron fluxes** used to re-normalize 1-D fluxes for all components
 - **Irradiation time:** 3 FPY FW, 9 FPY Blanket-Cell I, 40 FPY other components
 - **Continuous operation** (need availability to run pulsed case)
- **LiPb/SiC System:**
 - **SiC** structure generates **very low afterheat** compared to FS and V
 - **LiPb** generates **higher afterheat** than SiC
 - In ARIES-AT, **LOFA is more critical than LOCA**
 - Afterheat calculation is done
 - Waste disposal and clearance analyses are in progress

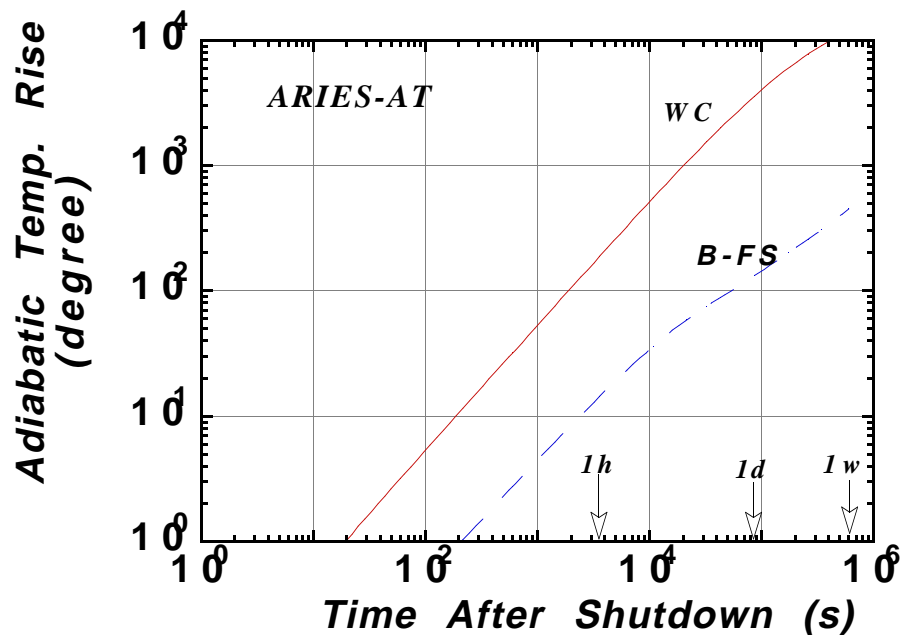


Afterheat Comparison Between WC and FS-based IB HT Shield



WC reduces radial build by ~5 cm but generates higher afterheat than B-FS

Temperature Rise in IB HT Shield During LOCA/LOFA



- **Adiabatic** calculations indicate **excessive temp rise in WC**-based IB HT shield after onset of LOCA/LOFA
- **Realistic LOCA/LOFA** analysis results in **lower temp rise**
- **B-FS** filler is recommended for IB HT shield
- **What is the max. allowable temp. for SiC** ($T_m = 2700\text{ C}$) during LOCA/LOFA?

Clearance Issues

ARIES-RS Design

Components	Volume (m ³)	Clearance Index	Cleared?
Blanket (compact)	25 (2%)	>> 1	no
Shield	560 (46%)	>> 1	no
Vacuum Vessel	175 (15%)	> 1	no
Magnet	440 (37%)	< 1	yes

Magnets (~35%) are always cleared

Blanket and shield (~50%) of all fusion designs will never meet clearance requirement

V.V. could be cleared with thicker shield

Clearance Issues

For VV, clearance limit is more restrictive than reweldability limit

Option I:

Meeting VV reweldability requirement (1 appm He)

⇒ Thin shield not cleared, VV not cleared

$$\text{Shield volume} = V_1$$

$$\text{VV volume} = V_2$$

$$\text{Magnet volume} = V_3$$

Option II:

Meeting VV clearance requirement

- Need 20-30 cm thicker shield
- Thicker shield not cleared, VV cleared
- Larger non-cleared shield volume

$$\text{Shield volume} > V_1$$

$$\text{VV volume} > V_2$$

$$\text{Magnet volume} > V_3$$

⇒ Larger “waste + cleared” volume !

If incremental increase in shield thickness is comparable to V.V. thickness, dispose of V.V. along with shield as Class C radwaste

Option III:

Meeting VV clearance requirement

- Need 20-30 cm thicker shield
- Design thin V.V. (~10 cm thick)
- Thicker shield not cleared, thin VV cleared
- Larger non-cleared shield volume

$$\text{Shield volume} > V_1$$

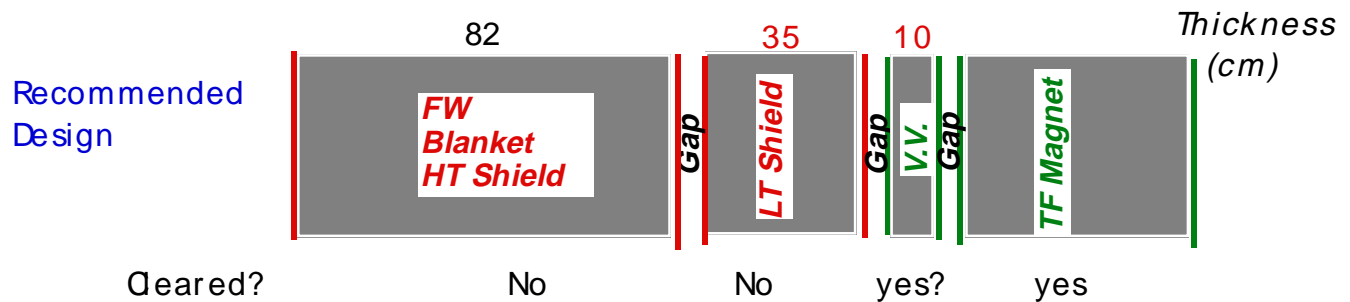
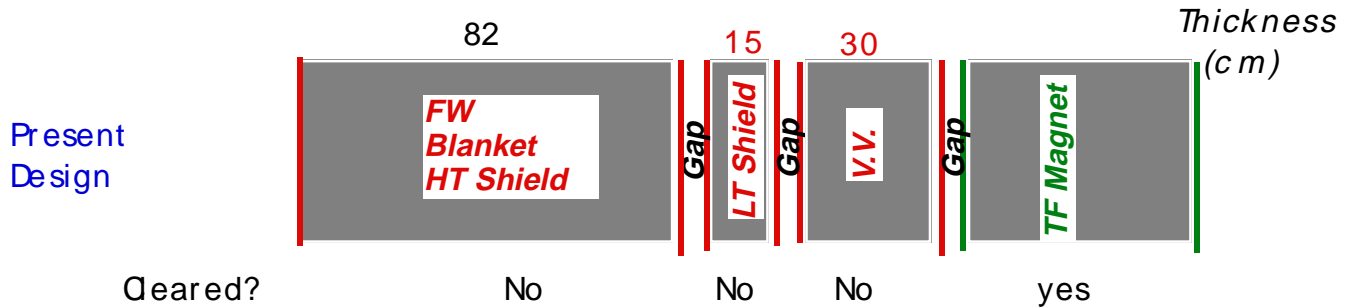
$$\text{VV volume} < V_2$$

$$\text{Magnet volume} \sim V_3$$

⇒ ~ same “waste + cleared” volume

Clearance Issues (cont.)

ARIES- AT Outboard Components



Activation analysis will determine boundary between radwaste and cleared components

How thin the V.V. could be? In ARIES-AT, 10 cm thick V.V. may qualify as cleared component

Shield should help clear as many components as design allows w/o significant increase in radwaste volume or cost