

# Views on Neutronics and Activation Issues Facing Liquid-Protected IFE Chambers

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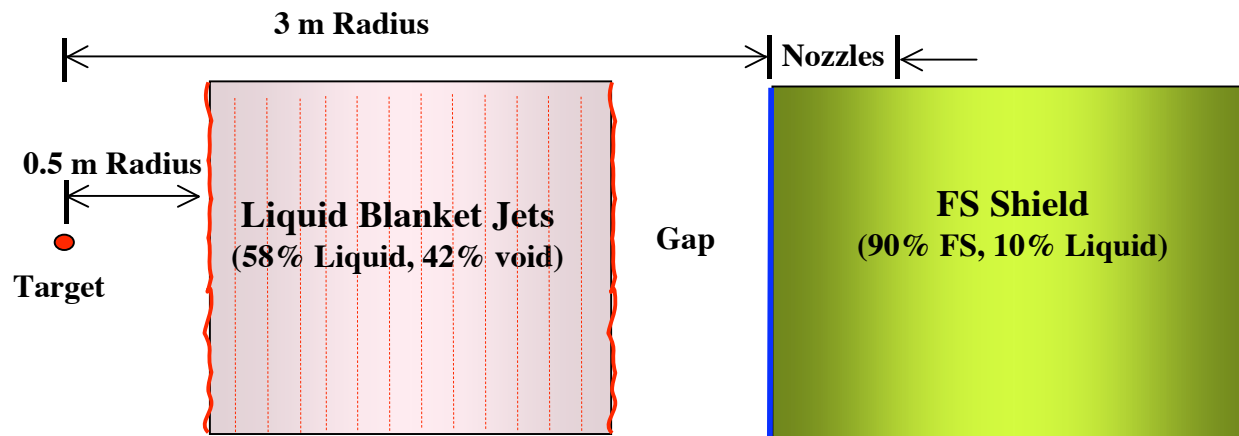
Fusion Technology Institute  
University of Wisconsin - Madison

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# Objectives

- **Develop design space and operational windows for ARIES-IFE-HIB (no point design).**
- **Concerns:**
  - **Breeding** potential of candidate breeders: Flibe & Flinabe
  - Ability of liquid wall to **protect structure** for 40 FPY
  - **Activation level** of structural components: shield & nozzles
  - **Isochoric heating** problems
  - Effect of radiation damage and cyclic fatigue on **structure lifetime.**

# Schematic of Radial Build



- **Flibe** ( $\text{BeF}_2, (\text{LiF})_2$ ) and **Flinabe** ( $\text{NaF}, \text{LiF}, \text{BeF}_2$ ) with natural Li.
- **ODS FS** (preferred structure) or **304-SS**.
- Innermost layer of shield represents **nozzles** and feeding tubes.
- Point source and **1-D spherical** geometry.



# Key ARIES-IFE-HIB Parameters

<b>Target yield</b>	<b>460 MJ</b>
<b>Rep rate</b>	<b>4 Hz</b>
<b># of pulses</b>	<b>126 million/FPY</b>
<b>Average source neutron energy</b>	<b>11.8 MeV</b>
<b>Penetrations coverage</b>	<b>3%</b>
<b>Plant lifetime</b>	<b>40 FPY</b>
<b>Availability</b>	<b>85%</b>



# ARIES-IFE Requirements and Design Limits

**Overall TBR**

$\geq 1.08$

**dpa\*** to structure

$\leq 200$  dpa for FS

$\leq 25$  dpa for 304-SS

**He production** for reweldability of FS

$\leq 1$  He appm

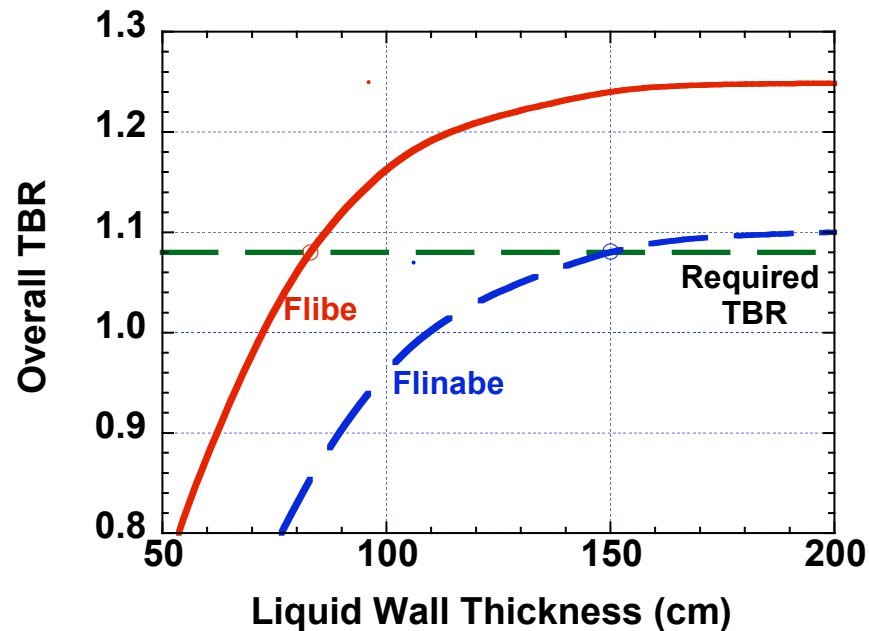
**WDR** for Class C low level waste

$\leq 1$

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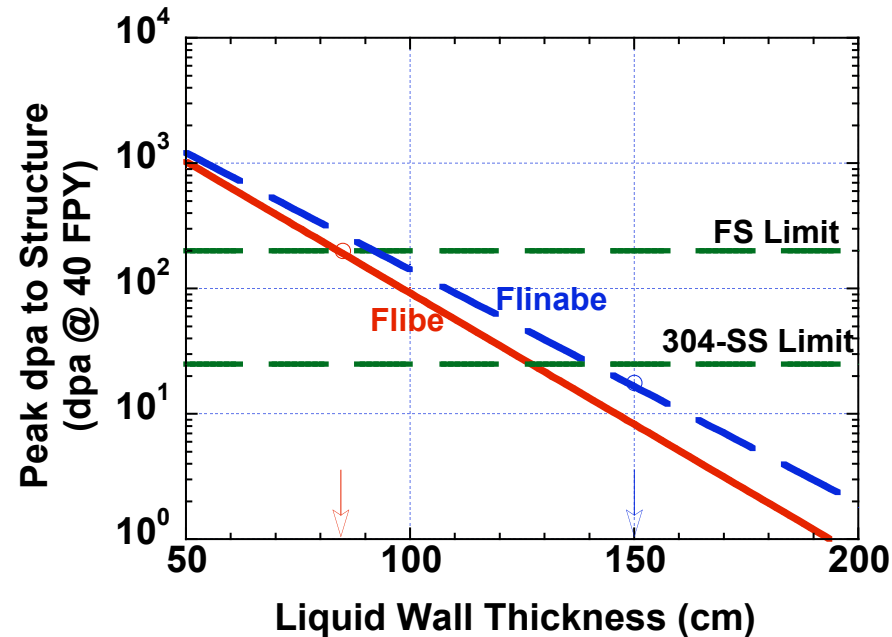
\* Cyclic fatigue could be more restrictive life-limiting factor than radiation damage.

# Flibe Breeds more Tritium than Flinabe



- 85 cm thick Flibe and 150 cm thick Flinabe meet breeding requirement.
- Enrichment does not enhance breeding of thick Flinabe.
- Nuclear energy multiplication amounts to ~1.25.

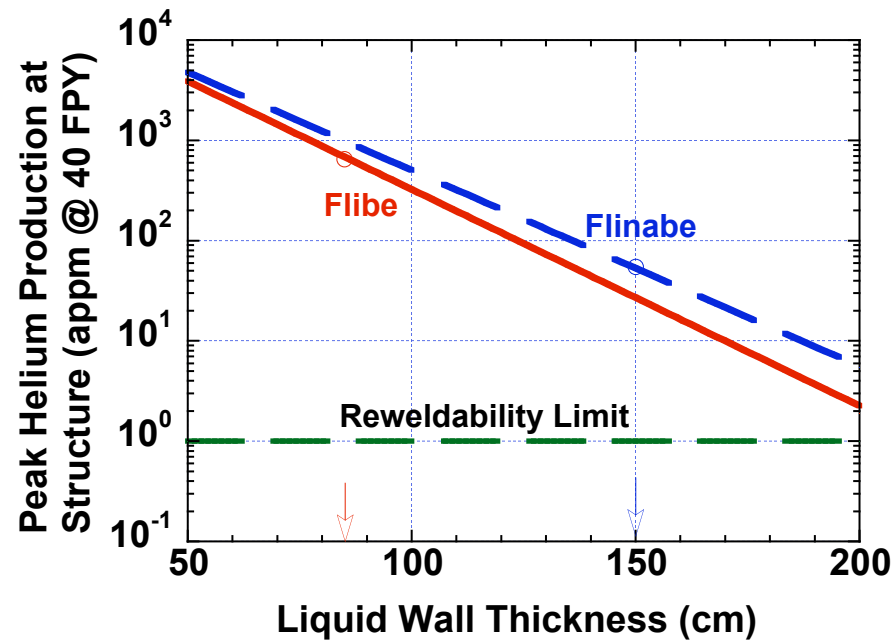
# Flibe has Slightly Better Shielding Performance than Flinabe



- 85 cm **Flibe** blanket meets 200 dpa limit for advanced FS only.
- 1.5 m **Flinabe** meets dpa limits for both structures.



# Excessive Helium Production at Chamber Structure



**Problem:** Innermost layer of shield and nozzles cannot be rewelded at any time during operation.



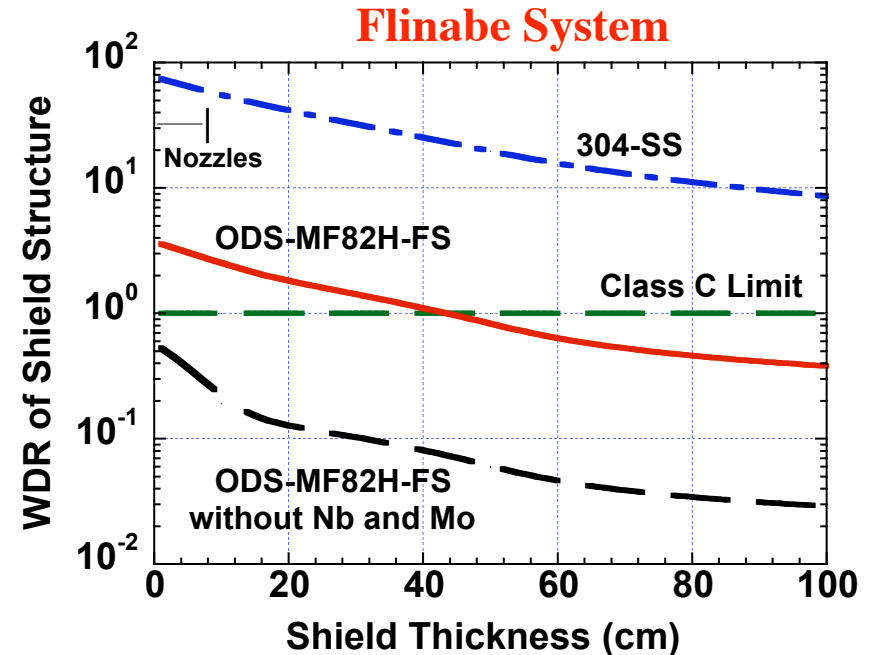
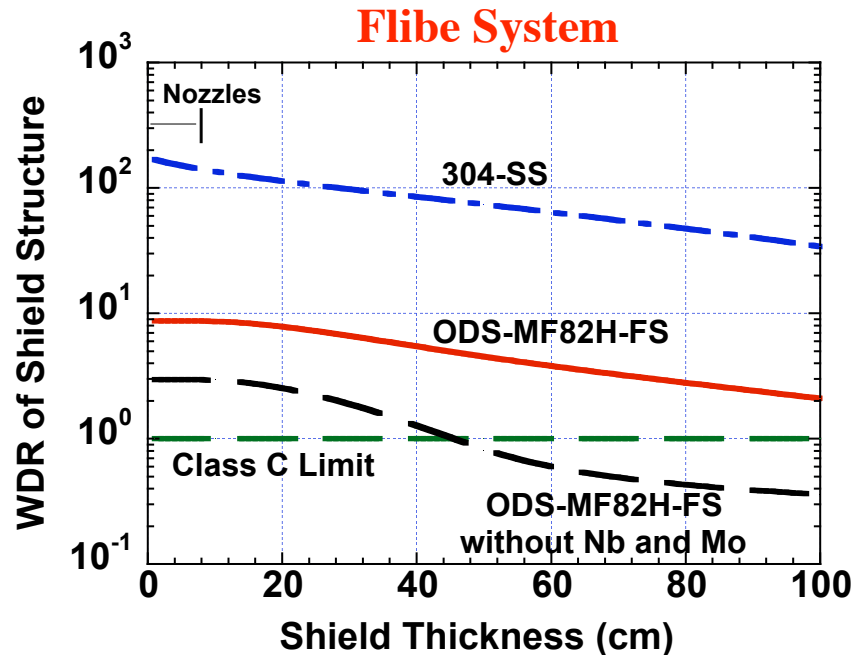
# Steel Composition (in wt%)

	<u>ODS M-F82H-FS*</u>	<u>304-SS#</u>
Fe	87.891	70.578
C	0.04	0.046
N	0.005	0.038
O	0.13	—
Si	0.24	0.47
P	0.005	0.026
S	0.002	0.012
Ti	0.09	0.03
V	0.29	—
Cr	8.7	17.7
Mn	0.45	1.17
Co	0.0028	0.1
Ni	0.0474	9.3
Cu	0.01	0.2
Nb	0.00033	—
Mo	0.0021	0.33
Ta	0.08	—
W	2	—
Y	0.7	—

\* IEA Modified F82H FS + 0.25wt% Y<sub>2</sub>O<sub>3</sub>, per M. Billone (ANL). Other elements include: B, Al, As, Pd, Ag, Cd, Sn, Sb, Os, Ir, Bi, Eu, Tb, Dy, Ho, Er, U.

# C. Baker et al., "Starfire-A Commercial Tokamak Fusion Power Plant Study," Argonne National Laboratory Report, ANL/FPP-80-1 (1980).

# All Steel Alloys Generate High Level Waste



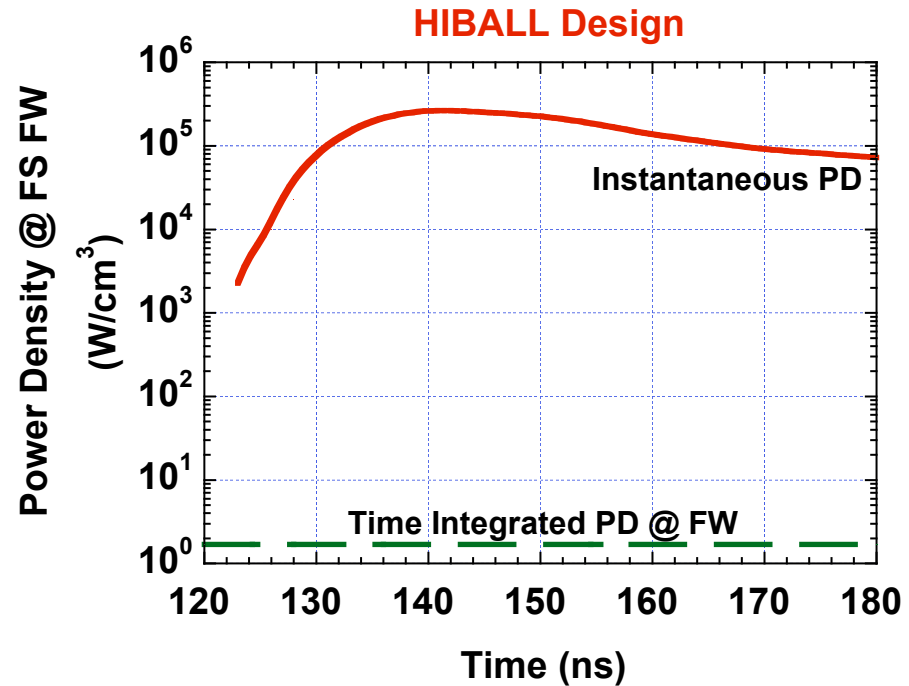
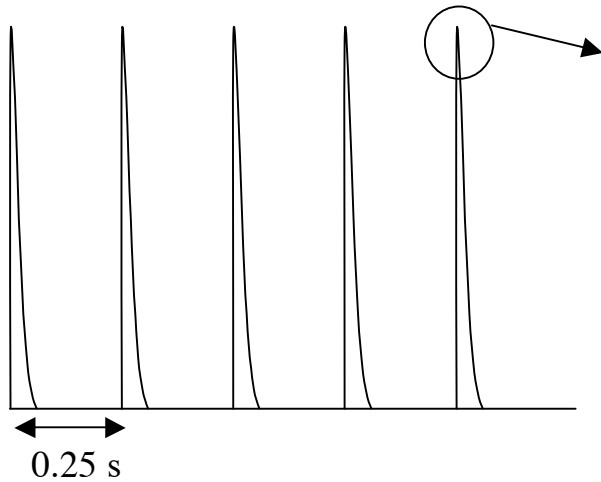
- 304-SS generates very high level waste.
- Main contributors to WDR:  $^{94}\text{Nb}$  (from Nb),  $^{99}\text{Tc}$  (from Mo), and  $^{192\text{n}}\text{Ir}$  (from W).



# Potential Solutions for Waste Problem

- **Thicken blanket** and **deplete** Flibe/Flinabe (**cost?**),
  - Average WDR over **thicker shield** (> 50 cm),
  - **Control Mo and Nb** for Flibe system in particular (**cost?**).
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- In practice, **Mo and Nb impurities cannot be zeroed out**. Actual level depends on \$/kg to keep Mo and Nb  $\ll$  1 wppm.
  - **Nozzles generate high level waste** unless mixed with shield and disposed as single unit at end of life.

# Isochoric Heating



- FS **temperature** fluctuates 4 times per second.
- Nuclear heating will induce **stresses** on the order of **10 MPa** in FS
- **Fatigue** from cycling and repetitive shock wave could:
  - Cause internal cracks
  - Shorten structure life
- When combined with radiation damage, **fatigue life could be more restrictive than 200 dpa limit.**

# Concluding Remarks

- No breeding problem identified for Flibe and Flinabe.
- Excessive helium production at structure precluding FS reweldability during operation.
- Steel-based structure produces high level waste ( $WDR \gg 1$ ), mandating:
  - Thicker blanket with depleted lithium (cost ?)
  - Shield  $> 50$  cm thick, and/or
  - Nb and Mo impurity control (cost?).
- Nozzles need additional protection to qualify as low level waste unless mixed and disposed with shield.
- Combined effect of radiation damage and fatigue on structure lifetime should be addressed in future studies.