

Status of Advanced Design Studies and Overview of ARIES-AT Study

Farrokh Najmabadi

US/Japan Workshop on
Fusion Power Plant Studies & Advanced Technologies
with E.U. Participation

March 16-18, 2000

UC San Diego

Electronic copy: <http://aries.ucsd.edu/najmabadi/TALKS>

ARIES Web Site: <http://aries.ucsd.edu/ARIES>

Role of Fusion in a Sustainable Global Energy Strategy

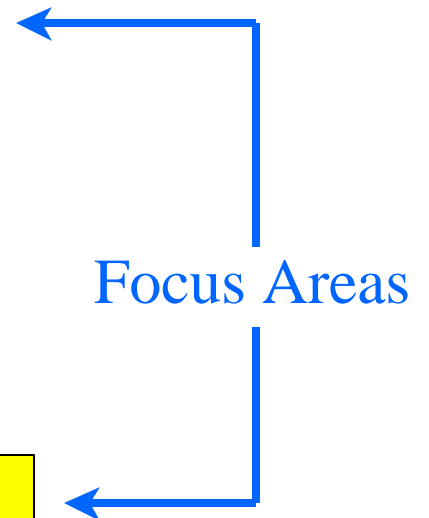
- Most of the socioeconomic studies were launched in FY 99:
 - * Study of options to deploy large fusion power plant including hydrogen production and co-generation. (ORNL & Partners). Completed in 12/99. (Presentation by L. Waganer)
 - * Establish the merits and address issues associated with fusion implementation (PPPL). (Presentation by J. Schmidt)
 - * Macro-economics modeling of global energy market and role of fusion (PNL) (Continuation of previous work).
 - * Comparison of various sources of energy based on equivalent CO₂ emission (U. Wisc.). (Presentation by J. Kulckinski)

National Power Plant Studies Program Initiated Two-years Projects in 1/99

- **Fusion Neutron Source Study:**
 - Non-electric applications of fusion, specially those resulting in near-term products may lead to new clients and to additional resources for fusion.
 - A concept definition study was performed to identify promising concepts and provide necessary information for proceeding further.
- **ARIES-AT:**
 - Assess impact of advanced technologies as well new physics understanding & modeling capabilities on the performance of advanced tokamak power plants.
- **Integrated IFE Chamber Study: (to start in 4/2000)**
 - Identify and explore design window for IFE chambers.

Non-Electric Applications of Fusion Neutrons

- Typical applications ($\sim 10^{19}$ - 10^{21} n/s) :
 - * Transmutation of fission waste (Actinides);
 - * Hybrids for fuel and/or energy production;
 - * Fusion materials and engineering testing.
- Post-cold-war additions:
 - * Tritium production;
 - * Burning of plutonium from dismantled weapons.
- Recent application ($\sim 10^{11}$ - 10^{13} n/s)
 - * Radioisotope production;
 - * Medical radiotherapy;
 - * Detection of explosives.



Key Conclusions of Neutron Source Study

- There are many different transmutation fuel cycles and many different blankets proposed.
- There is no established set of criteria. Most concepts can be re-optimized based on different criteria, so comparison is difficult.
- Work by other communities has focused on performance (cost, electricity production, burn through) not on safety, licensing, reliability, cost, etc.
- Most performance parameters mainly depend on blanket and fuel cycle choices (spectrum of external source matters little).

Key Conclusions of Neutron Source Study

- The most fundamental distinction is the existence of an external neutron source leading to critical versus sub-critical blanket operation.
 - * Fission (near-term technology)
 - * Fusion & accelerator (sub-critical operation and deeper burn)
- The cost of the external neutron source is an added cost. Trade-off of sub-critical operation and deeper burn versus cost difficult in the absence of established criteria. Compared to accelerators, fusion has the added difficulty of the need to breed tritium.
- This is NOT a near-term option for fusion because of the safety and licensing issues and the associated need for component data base and reliability. Same is true for accelerator-based systems.
- Both ATW and fusion-based system are technically viable long-term options.

ARIES-RS Study Sets the goals and Direction of Research for ARIES-AT

	<u>ARIES-RS Performance</u>	<u>ARIES-AT Goals</u>
Economics		
Power Density	Reversed-shear Plasma Radiative divertor Li-V blanket with insulating coatings	Higher performance RS Plasma, High T_c superconductors
Efficiency	610° C outlet (including divertor) Low recirculating power	> 1000° C coolant outlet > 90% bootstrap fraction
Availability	Full-sector maintenance Simple, low-pressure design	Same or better
Manufacturing		Advanced manufacturing techniques
Safety and Environmental attractiveness	Low afterheat V-alloy No Be, no water, Inert atmosphere Radial segmentation of fusion core to minimize waste quantity	SiC Composites Further attempts to minimize waste quantity

Main Features of ARIES-AT²

(Advanced Technology & Advanced Tokamak)

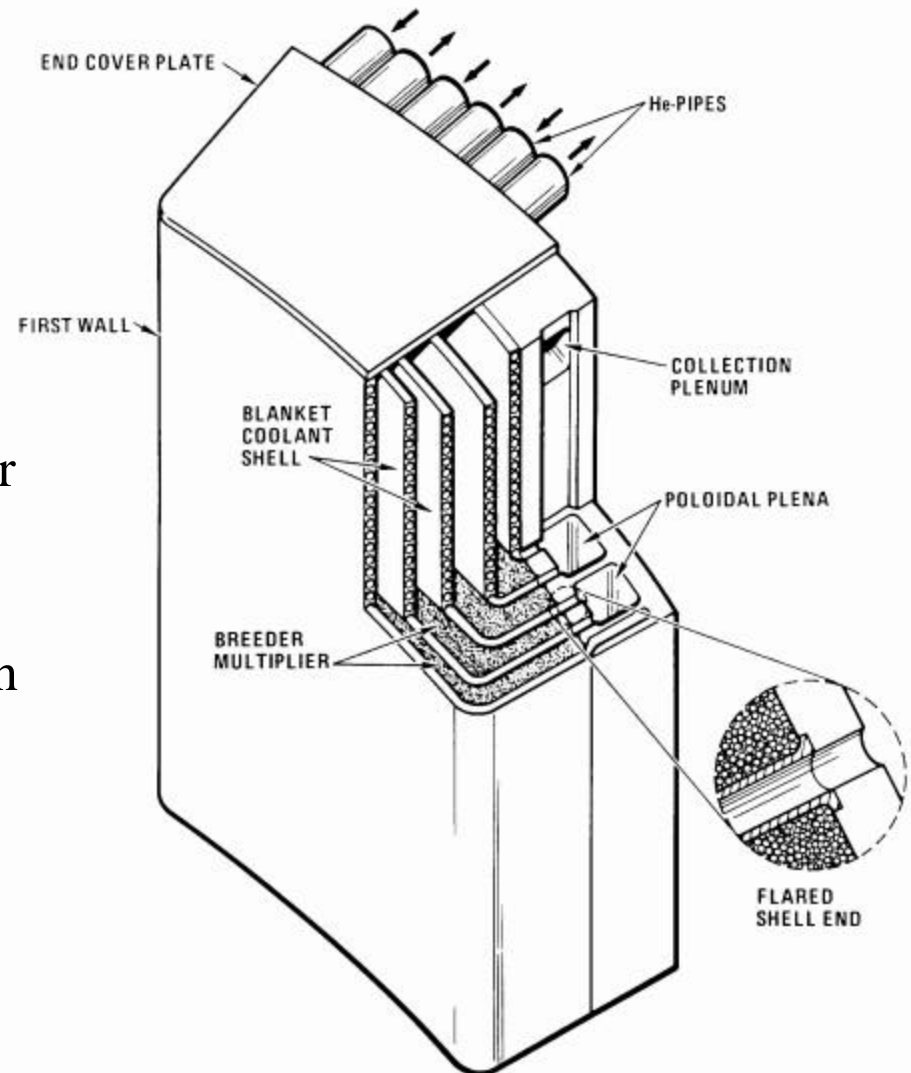
- **High Performance Very Low-Activation Blanket:** New high-temperature SiC composite/LiPb blanket design capable of achieving ~60% thermal conversion efficiency with small nuclear-grade boundary and excellent safety & waste characterization.
 - **Higher Performance Physics:** reversed-shear equilibria have been developed with up to 50% higher β than ARIES-RS and reduced current-drive power.
 - **Higher Performance Magnets:** High- T_c superconductors.
- ⇒ Present strawman operates at the same power density as ARIES-RS, higher β was used to reduce the peak field at the magnet.
- Reduce unit cost of components through **advanced manufacturing techniques.**

ARIES-AT²: Physics Highlights

- Use the lessons learned in ARIES-ST optimization to reach a higher performance plasma;
 - Using $> 99\%$ flux surface from free-boundary plasma equilibria rather than 95% flux surface used in ARIES-RS leads to larger elongation and triangularity and higher stable β .
- Eliminate HHFW current drive and use only lower hybrid for off-axis current drive.
- Perform detailed, self-consistent analysis of plasma MHD, current drive and divertor (using finite edge density, finite p' , impurity radiation, etc.)
- ARIES-AT blanket allows vertical stabilizing shell closer to the plasma, leading to higher elongation and higher β .

ARIES-I Introduced SiC Composites as A High-Performance Structural Material for Fusion

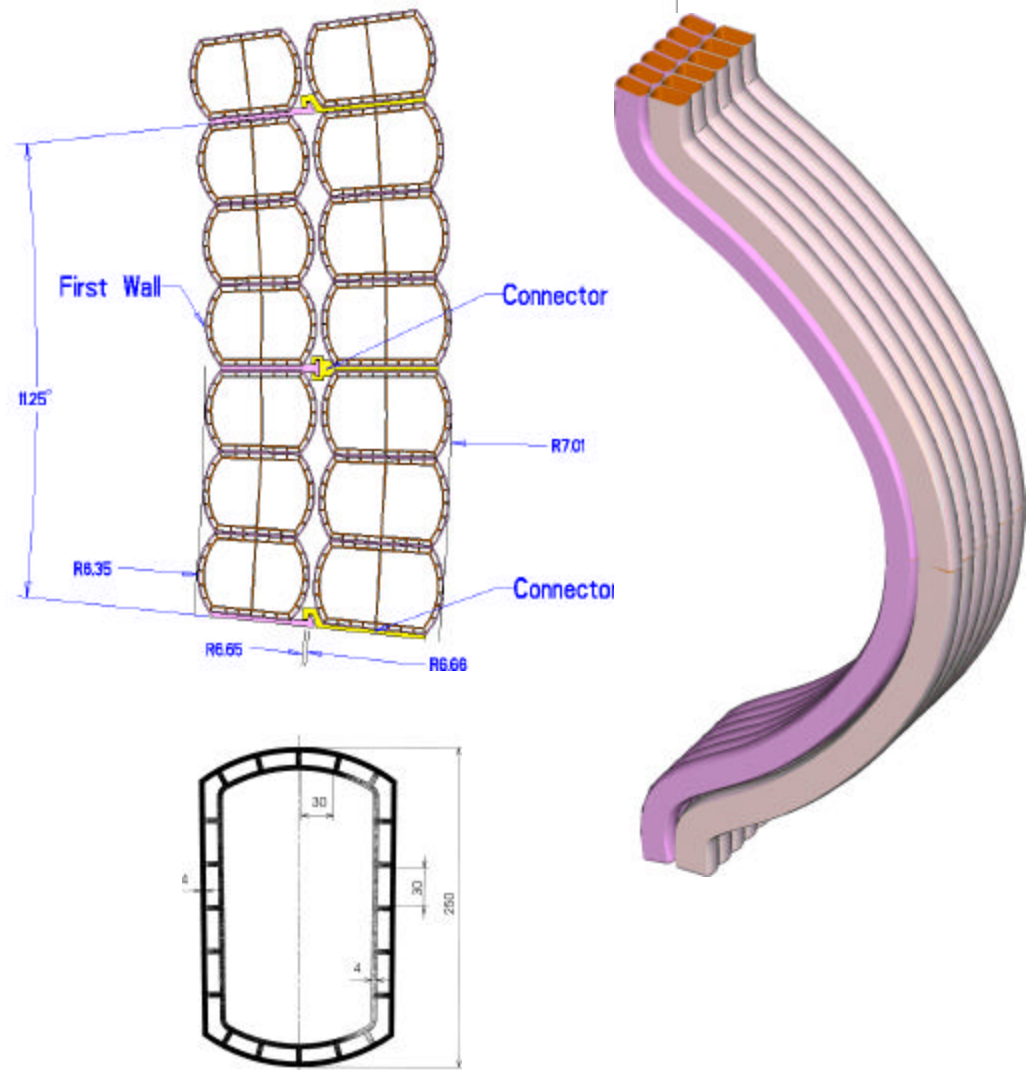
- Excellent safety & environmental characteristics (very low activation and very low afterheat).
- High performance due to high strength at high temperatures ($>1000^{\circ}\text{C}$).
- Large world-wide program in SiC:
 - New SiC composite fibers with proper stoichiometry and small O content.
 - New manufacturing techniques based on polymer infiltration results in much improved performance and cheaper components.
 - Recent results show composite thermal conductivity (under irradiation) close to 15 W/mK which was used for ARIES-I.



ARIES-AT²: SiC Composite Blankets

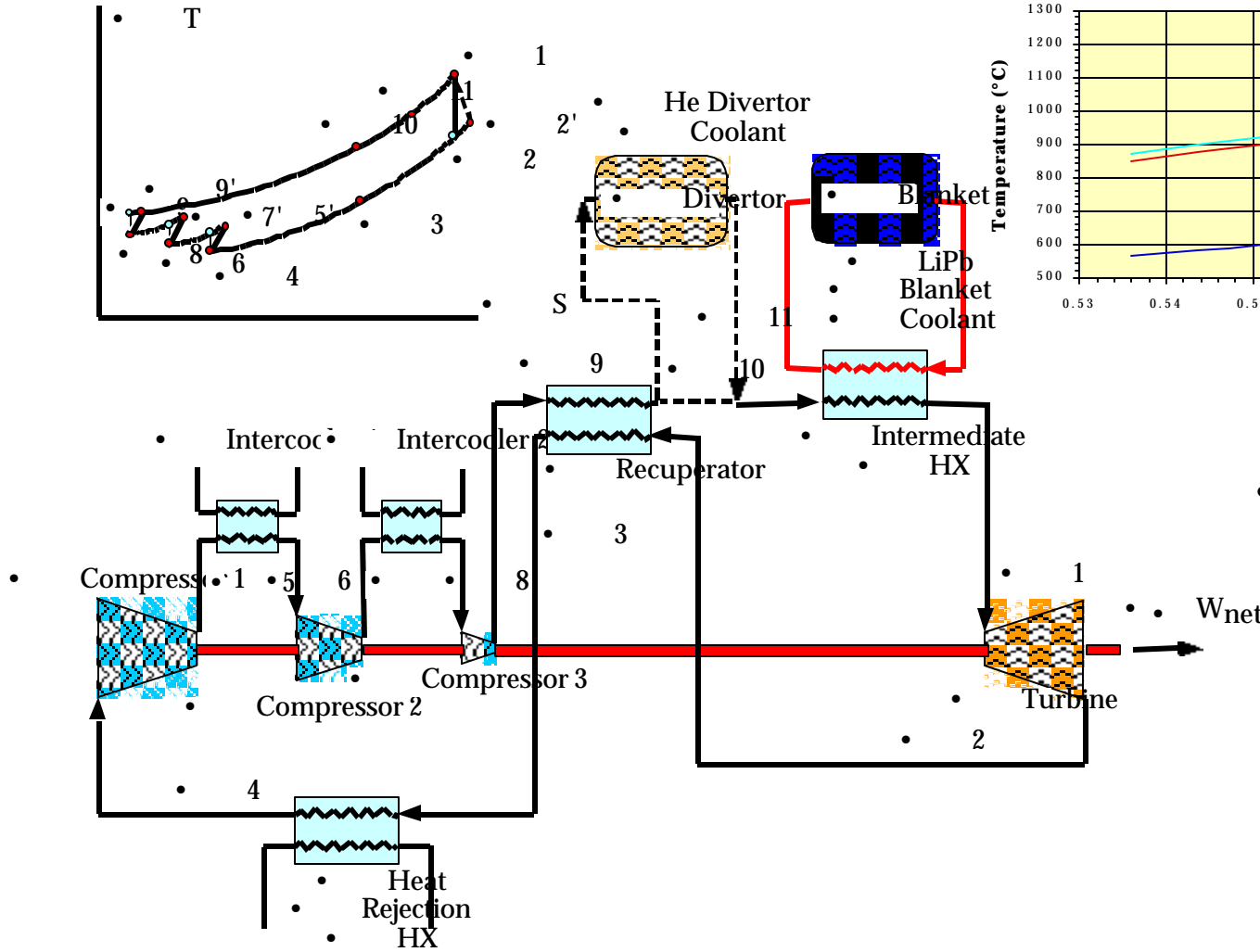
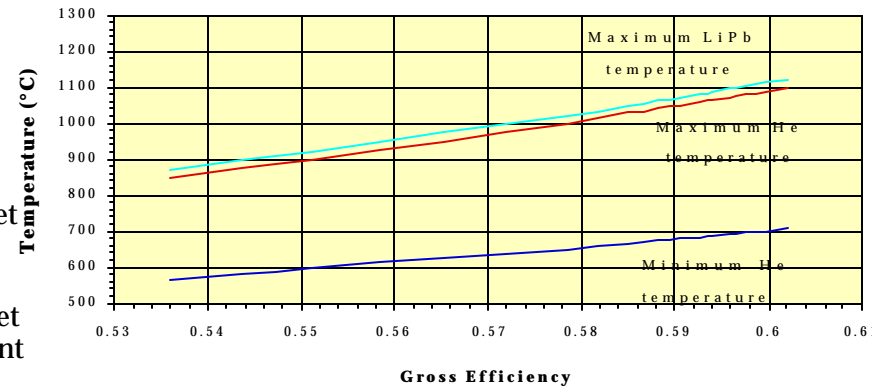
- Simple, low pressure design with SiC structure and LiPb coolant and breeder.
- High LiPb outlet temperature ($\sim 1100^{\circ}\text{C}$) and high thermal efficiency of $\sim 60\%$.
- Simple manufacturing technique.
- Very low afterheat.
- Class C waste by a wide margin.

Outboard blanket & first wall



Recent Advances in Brayton Cycle Leads to Power Cycles With High Efficiency

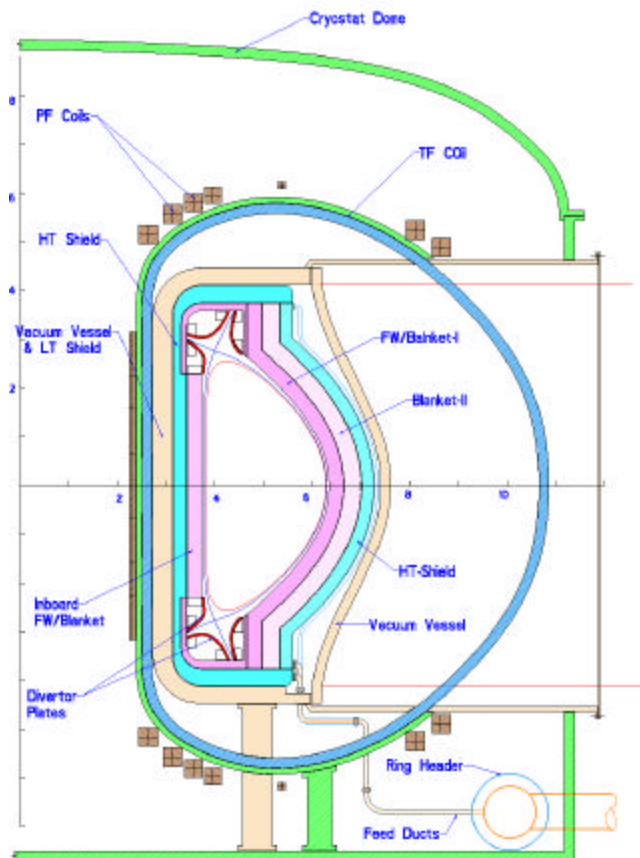
Brayton Cycle He Inlet and Outlet Temperatures as a Function of Required Cycle Efficiency



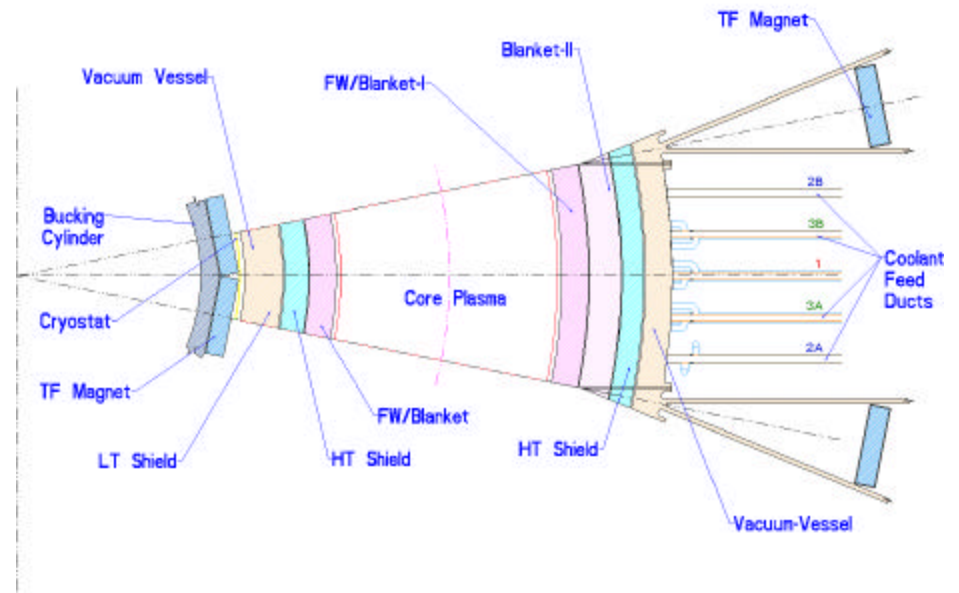
- Key improvement is the development of cheap, high-efficiency recuperators.

ARIE-AT also Uses A Full-Sector Maintenance Scheme

Cross-Section of ARIES-AT Power Core



Plan View of ARIES-AT Power Core(1/16 Sector)



Major Parameters of ARIES-RS and ARIES-AT

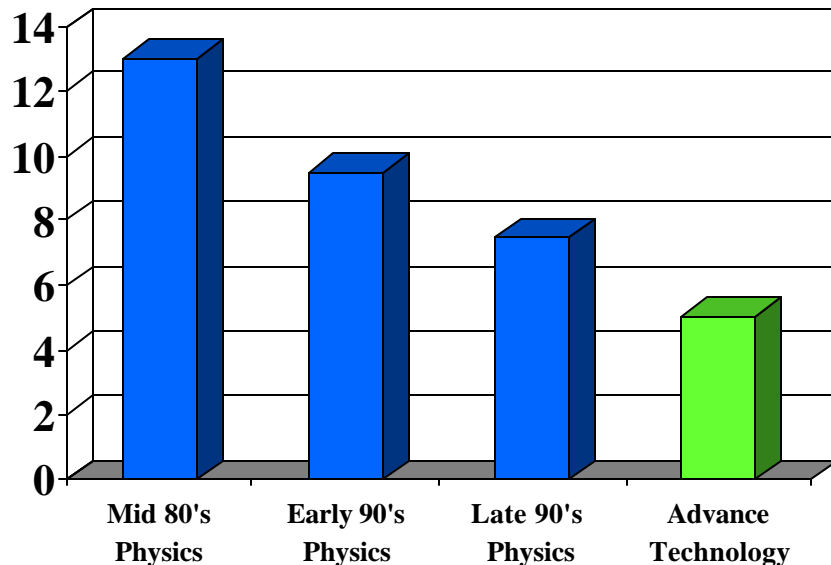
	ARIES-RS	ARIES-AT
Aspect ratio	4.0	4.0
Major toroidal radius (m)	5.5	5.2
Plasma minor radius (m)	1.4	1.3
Plasma elongation (κ_x)	1.9	2.2
Plasma triangularity (δ_x)	0.77	0.86
Toroidal β	5%	9.2%
Electron density (10^{20} m^{-3})	2.1	2.25
ITER-89P scaling multiplier	2.3	2.7
Plasma current	11	13

Major Parameters of ARIES-RS and ARIES-AT

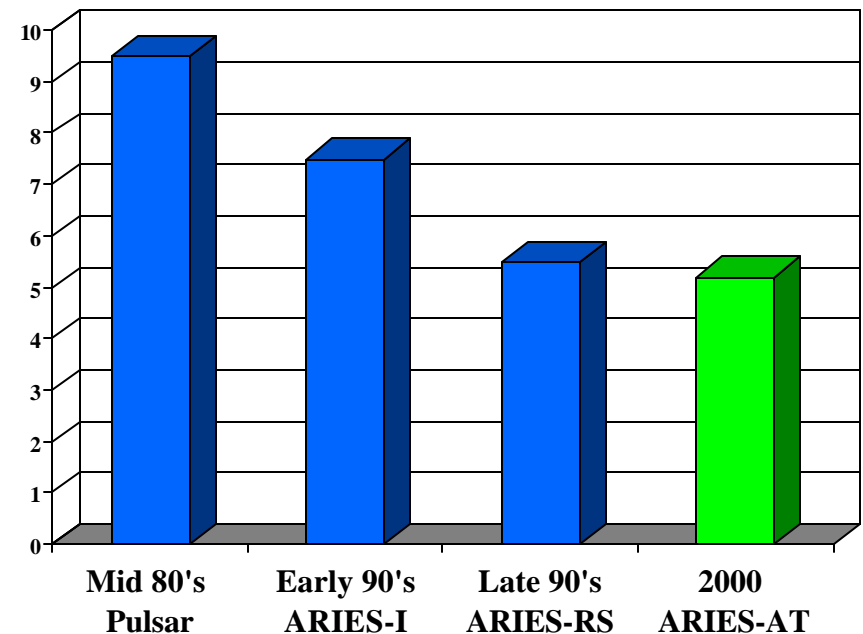
	ARIES-RS	ARIES-AT
On-axis toroidal field (T)	8	6
Peak field at TF coil (T)	16	11
Current-drive power to plasma (MW)	81	25
Peak/Avg. neutron wall load (MW/m ²)	5.4/ 4	4.7/3.8
Fusion power (MW)	2,170	1,720
Thermal efficiency	0.46	0.59
Gross electric power (MW)	1,200	1,136
Recirculating power fraction	0.17	0.12
Cost of electricity (mill/kWh)	76	53

Our Vision of Magnetic Fusion Power Systems Has Improved Dramatically in the Last Decade, and Is Directly Tied to Advances in Fusion Science & Technology

Estimated Cost of Electricity (c/kWh)



Major radius (m)

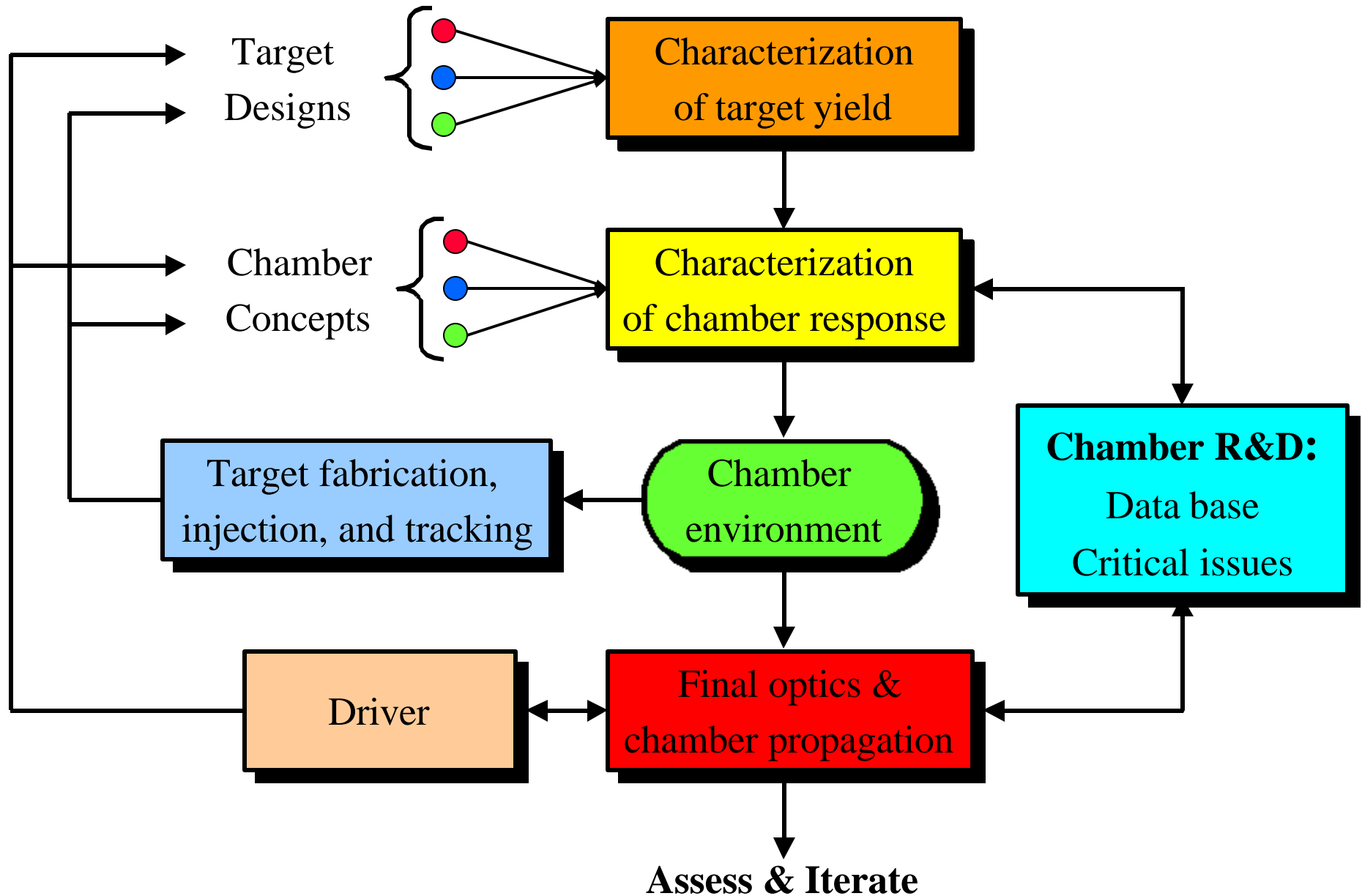


Present ARIES-AT parameters:

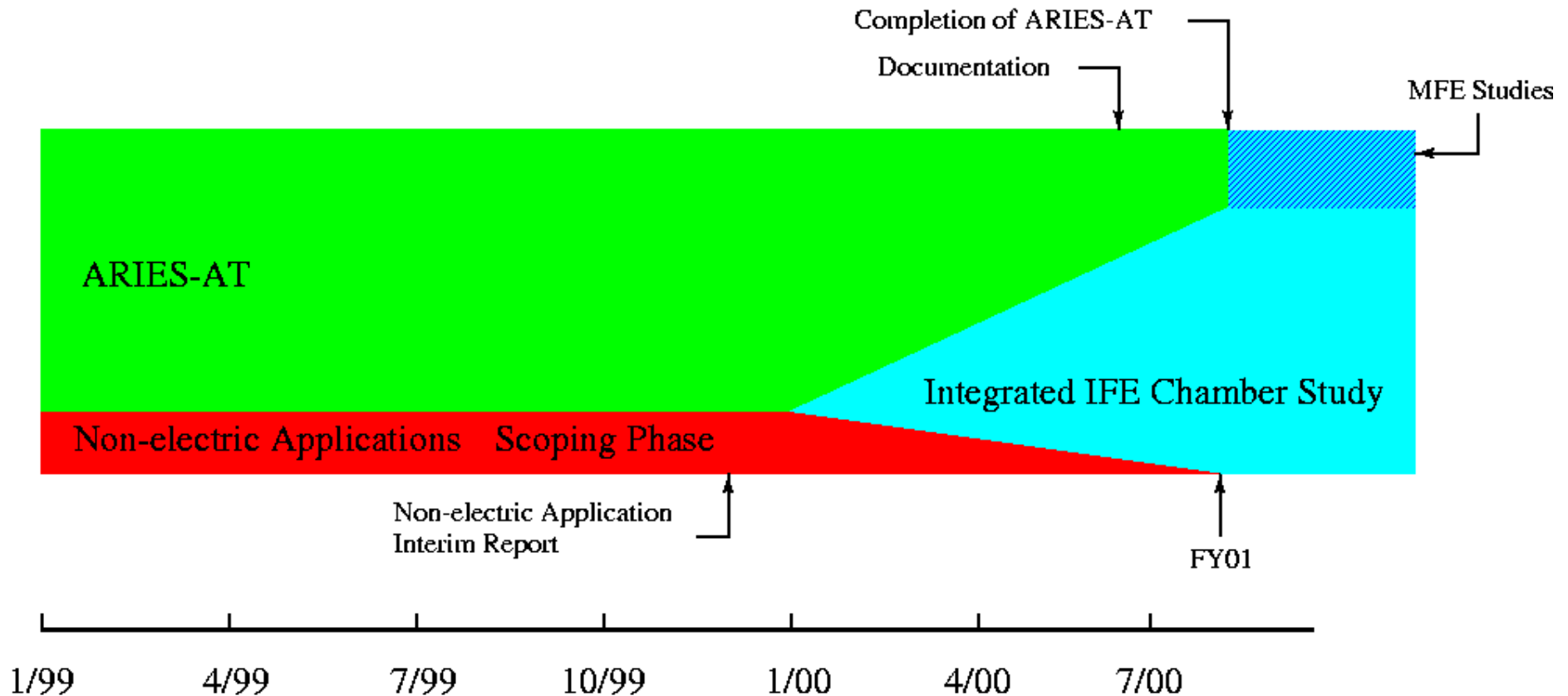
Major radius: 5.2 m
 Toroidal β : 9.2%
 Wall Loading: 4.75 MW/m²

Fusion Power 1,720 MW
 Net Electric 1,000 MW
 COE 5.3 c/kWh

The Integrated IFE Study Will Identify and Explore the Design Window for IFE chambers & Define R&D Needs



National Power Plant Studies Program Plan for FY 2000



- The scope and relative size of the MFE and IFE studies in FY01 depends to program budget and outcome of congressional process.